

<b>Technical Requirements for Electrical Equipment</b>  Title <b>Environmental Specification for Accident Conditions</b>	Document TBE 102:1
	Issue 9
	Date 2024-12-06
	Supersedes 8

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# 1 Introduction

This document specifies the environmental service conditions in addition to the specifications for normal operation as described in TBE 101. These additional conditions are applicable to electrical equipment that are required to be operational during and following Design Base Event (DBE). The equipment will be exposed to the harsh environment caused by the accident conditions.

It has to be demonstrated that the safety-related equipment will, at the end of its qualified life or end of its qualified condition, with margin be able to perform all safety related functions during accident conditions for the time required in the Technical Specification. The required time of operation during accident conditions is specified for each application.

It is degradation with time (ageing) followed by exposure to the environmental extremes of temperature, pressure, humidity, ionising radiation, vibration, and chemical spray resulting from Design Basis Events that presents a potential for causing common cause failures of safety related equipment. For this reason it is necessary to establish a qualified life or a qualified condition for equipment with significant ageing mechanisms. Ageing mechanisms will be handled in condition based qualification with condition measuring or by periodic surveillance/maintenance.

Examination procedure for environmental qualification are specified in KBE EP-154, which is based on IEC/IEEE 60780-323:2016 including some additional Swedish requirements.

## 2 Definitions

### Environmental Qualification

”Environmental qualification is a verification of design limited to demonstrating that the electric equipment is capable of performing its safety function under significant environmental stresses resulting from design basis accidents in order to avoid common-cause failures.”  
(Regulatory Guide 1.89, 1984)

### Qualified life

Period for which an equipment has been demonstrated, through testing, analysis and/or experience, to be capable of functioning within acceptance criteria during specific operating conditions while retaining the ability to perform its safety functions in accident condition or earthquake  
(IEC/IEEE 60780-323:2016)

### Condition-based qualification

”Qualification based on measurement of one or more condition indicators of equipment, its components, or materials for which an acceptance criterion can be correlated to the equipment’s ability to function as specified during an applicable design basis event” (IEC/IEEE 60780-323:2016)

## Service conditions

”Actual physical states or influences during the service life of equipment, including normal operating conditions, abnormal operating conditions, design basis event conditions and conditions following a design basis event and design extension conditions”

Note 1 to entry: In 2007-edition of IAEA safety glossary, accident conditions include both design basis accident and beyond design basis accident. This second notion has been replaced within IAEA by the notion of design extension conditions (AIEA, SSR-2/1). It explains the need of changing the definition.  
(IEC/IEEE 60780-323:2016)

## Beyond DBE

Very unlikely events Beyond Design Basis Events

## Environmental requirements

The environmental integrity requirements have been divided in two basic categories, depending on the installed location:

### Equipment installed outside the reactor containment

In rooms or compartments where the environmental conditions due to a possible high energy line break (HELB) have to be considered.

The postulated accident environment is plant specific and matched to the possibility of isolation of high-energy medium.

Swedish nuclear plants with similar accident profiles are grouped as follows:

<b>Profile 1-2</b>	Applicable to Oskarshamn 3
<b>Profile 3</b>	Applicable to Oskarshamn 3 and Ringhals 3-4
<b>Profile 4, General</b>	Applicable to all Swedish nuclear plants (Profile 4 according to KBE-EP 154) The Temperature shall exceed 60°C during at least 24h

Profiles 1-2 are built on automatic isolation on line break. Profile 3 is built on manual isolation on line break.

### Equipment installed inside the reactor containment

The postulated accident environment is plant specific. Swedish nuclear plants with similar accident profiles are grouped as follows:

<b>BWR 1</b>	Applicable to Forsmark 1-2
<b>BWR 2</b>	Applicable to Forsmark 3 and Oskarshamn 3
<b>BWR</b>	Generic accident profile that envelopes the BWR 1 and BWR 2 profiles above
<b>PWR</b>	Applicable to Ringhals 3 and 4

Electrical equipment that might be exposed to accident environment and that is required to be operational during/following the accident is assigned one of the plant group specific accident profiles above. If the equipment cannot be assigned to one of the above listed categories, the different requirements are specified.

## **3 Environmental Conditions**

### **3.1 General**

The environments for normal service conditions are described in TBE 101. The applicable equipment verification requirements for normal service conditions are specified in KBE EP-151.

This document specifies the service conditions during and following accident conditions. The applicable product verification requirements are specified in KBE EP-154.

The accident conditions are specified in Table 1 and 2. The applicable accident profile is specified in enclosed diagrams, one for each plant group.

The ionising radiation dose rate and accumulated dose is specified for each application depending on the installed location and the required time for operation during accident and post-accident conditions.

Optional environmental conditions may apply, depending on the required function or installed location, example: risk for submerging or rinsing. The applicable requirements for each type of equipment are specified in the Technical Specification.

Environmental conditions Beyond Design Basis Event is specified in TS.

## 4 Tables

### 4.1 Table 1 - Parameters for accident Conditions - Inside Reactor Containment

Location	Inside Reactor Containment				Remarks
Plant Group	BWR 1	BWR 2	BWR generic	PWR Ringhals	
<b>Temperature</b>	173°C 0-10 min 150°C 10 min- 120°C 6h 90°C 6-96 h >96 h	173°C 0-3 h 160°C 3-6 h 120°C 6-96 h 90°C >96 h	173°C 0-3 h 160°C 3-6 h 120°C 6-96 h 90°C >96 h	38-141°C 0-10 s 141°C 10 s-3 h 104°C 3 h-30 h 67°C >24 h	
<b>Pressure abs</b>	500kPa 0-6 h	500kPa 0-6 h	500kPa 0-6 h	210-514 kPa 0-10 s 514 kPa 10 s -3 h 238 kPa 3 h-24 h 160 kPa 24 h-7 days 134 kPa >7 days	
<b>Humidity</b>	100%, p<500kPa	100%, p<500kPa	100%, p<500kPa	100%	Saturated steam
<b>Ionising radiation</b>	To be specified for each application <b>Note 1</b>	To be specified for each application <b>Note 1</b>	To be specified for each application <b>Note 1</b>	To be specified for each application <b>Note 1</b>	
<b>Water Spray</b>	yes	yes	yes	yes	

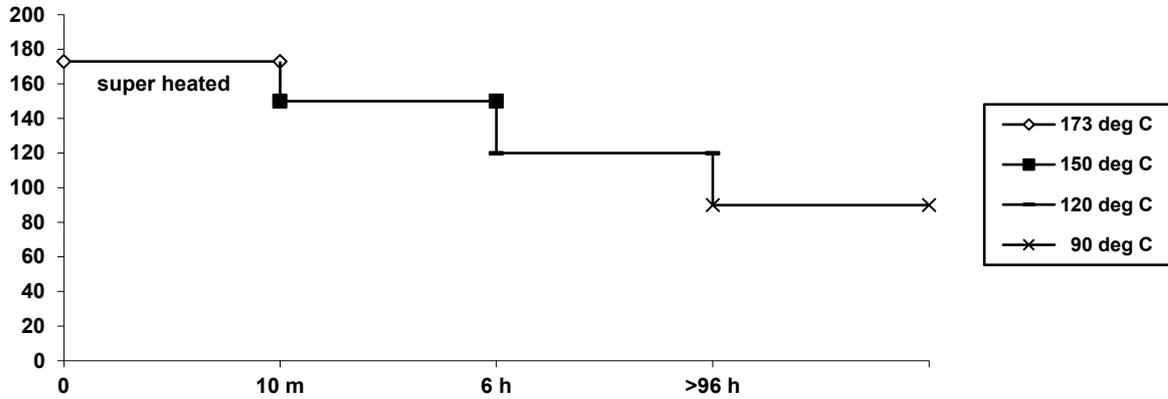
**Notes** 1) The ionising radiation dose rate and accumulated dose shall be specified for each application depending on the installed location and the required time for operation during accident and post-accident conditions.

## 4.2 Table 2 - Parameters for Accident Conditions - Outside Reactor Containment

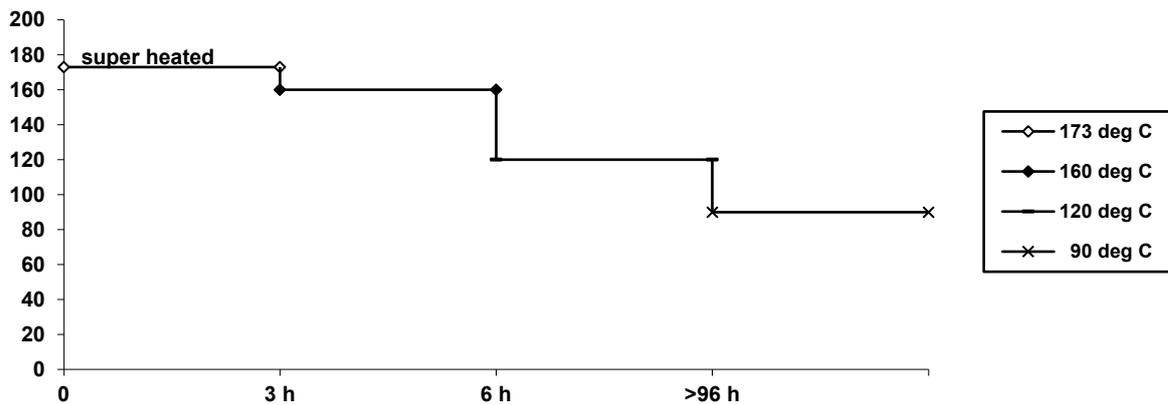
Location	Outside Reactor Containment							
Plant Group	Oskarshamn 3		Oskarshamn 3		Oskarshamn 3, Ringhals 3-4		BWR/PWR Generic, all plants	
	Profile 1		Profile 2		Profile 3		Profile 4, Generic	
Temperature	100°C	0-2 min	70°C	0-2 min	100°C	0-45 min	112°C	0-10 min
	100°C till 60°C	2 min-4h	70°C till 60°C	2 min-2h	100°C till 60°C	45 min-4h	100°C	10 min-3h
	60°C	4h-24h	60°C	2 h-24h	60°C	4 h-24h	60°C	>3 h
Pressure abs	150kPa	0-2 s			150kPa	0-2 s	150kPa	0-10 min
Humidity	100 %		100 %		100 %		100 %	
Radiation	To be specified for each application		To be specified for each application		To be specified for each application		To be specified for each application	
Water Spray	No		No		No		No	

## 5. Accident Profile

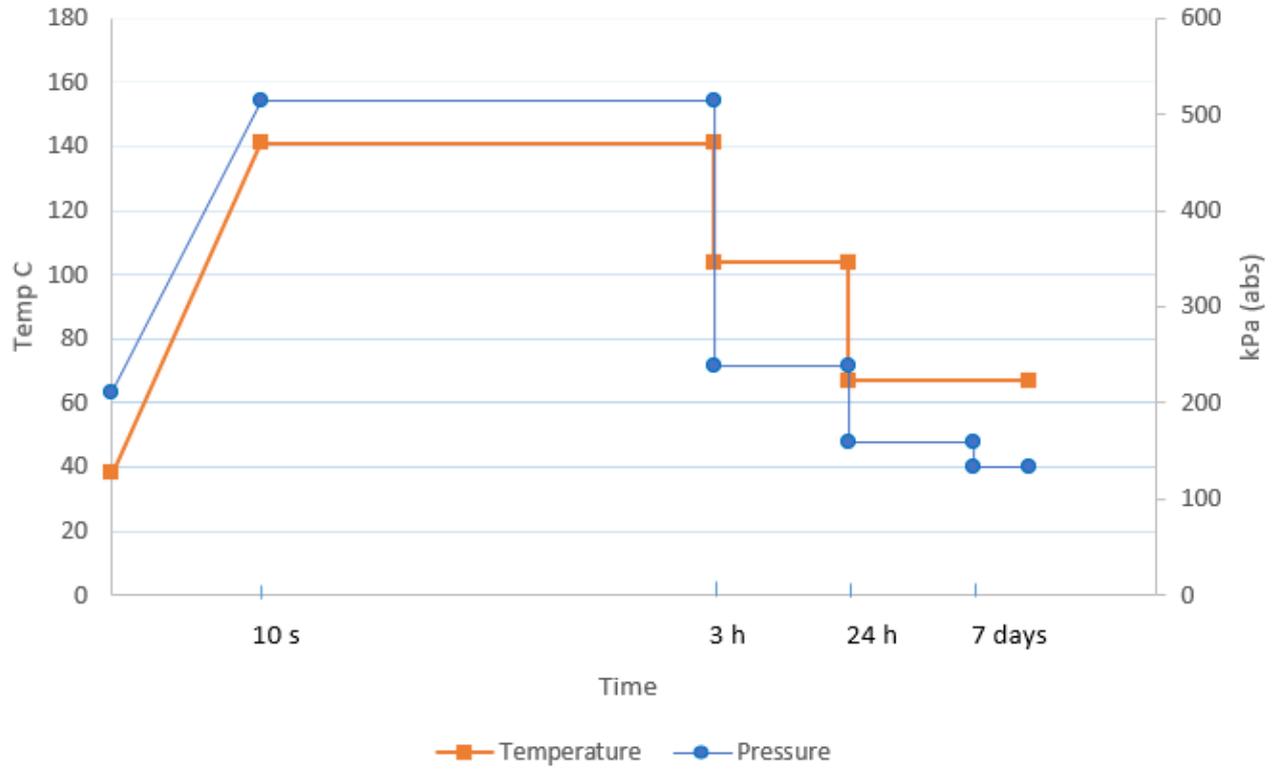
### 5.1 BWR 1 - Inside Containment Accident Profile



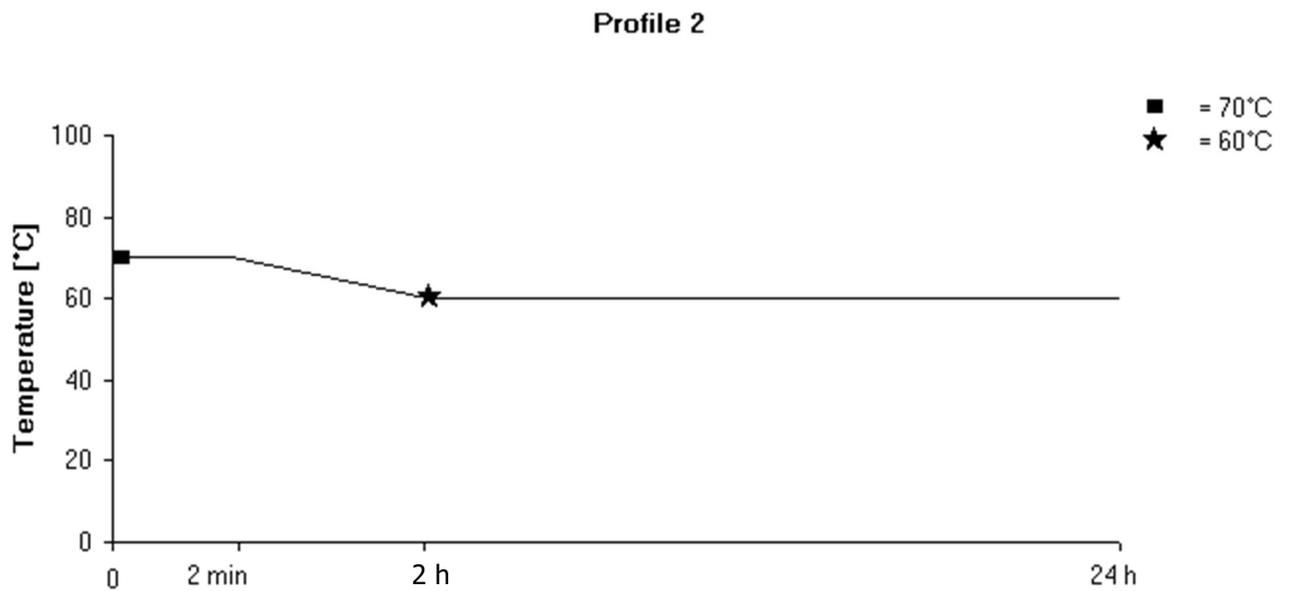
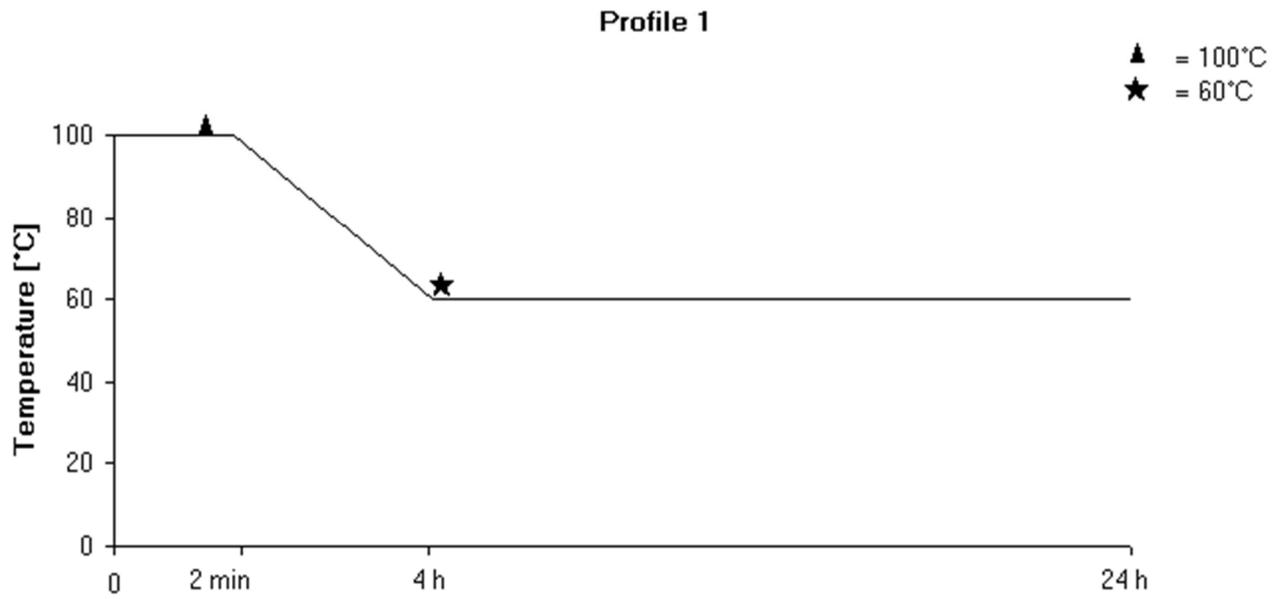
### 5.2 BWR 2 / BWR generic - Inside Containment Accident Profile



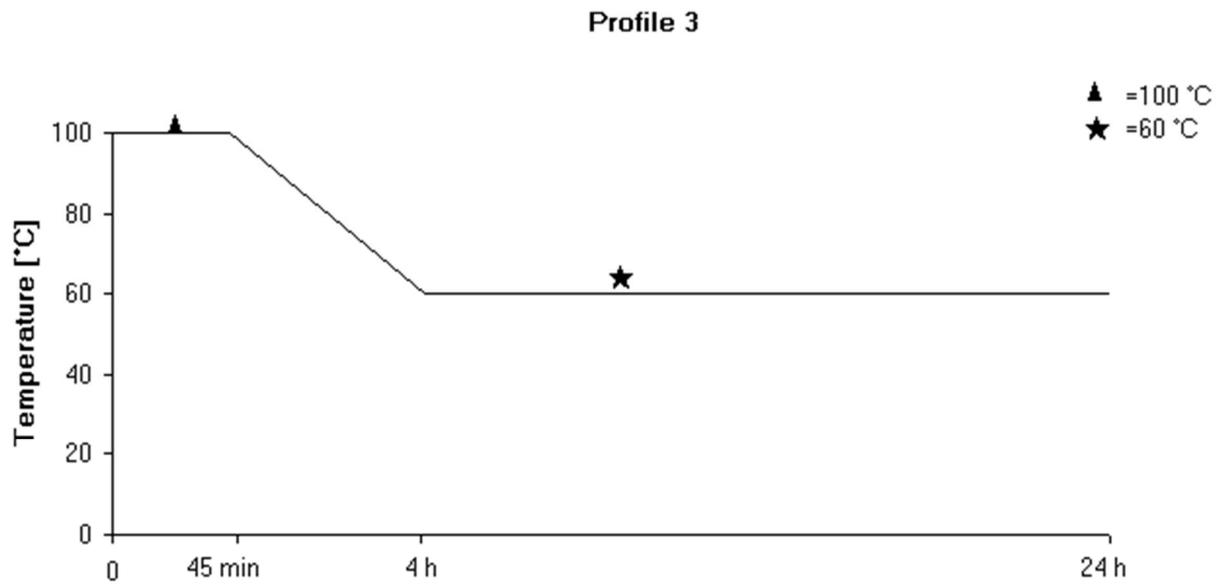
### 5.3 PWR - Inside Containment Accident Profile



## 5.4 Oskarshamn 3 - Outside Containment Accident Profiles



## 5.5 Oskarshamn 3 and Ringhals 3-4 - Outside Containment Accident Profile



## 5.6 Generic: - Outside Containment Accident Profile

